

## 机械与动力工程学院博士生资格考试笔试大纲

## Syllabus of Ph.D. Qualification Examination (SJTU-ME)

*笔试主题 Exam Topic	(中文) 反应堆材料学
	(English) Nuclear Reactor Materials
*考核形式 Exam Format	闭卷考试, 1 小时 Closed-book exam, 1 hour
*考核目标 Exam Target	<p>(1) 考察研究生对反应堆材料基础知识掌握的程度, 重点考察水冷对材料方面的内容;</p> <p>(2) 分析问题解决问题的能力;</p> <p>(3) 新型反应堆及关键核级材料未来的发展趋势。</p> <p>(1) Examine the basic knowledge of reactor materials , emphasizing on the materials for water cooled power reactors;</p> <p>(2) Ability to analyze and solve problems;</p> <p>(3) The future development trend of new reactor design and key nuclear grade materials.</p>
*考核内容 Exam Contents	<p>(1) 材料的成分、晶体类型、组织结构与材料性能的关系; The correlation of materials performances with chemical compositions, crystal structure and microstructures.</p> <p>(2) 金属材料的强化机理、典型压水堆核电厂主要设备、堆内结构所用的材料及其性能特点; Strengthening of materials, materials for PWR power reactors, including nuclear grade equipment such as primary piping, pressure vessels and internals.</p> <p>(3) 有机、无机氧化物、金属等工程材料在 <math>\gamma</math> 射线和快中子辐照条件下的辐照损伤机理及对材料性能的影响规律; Radiation damage to reactor materials.</p> <p>(4) 金属材料在核电厂高温高压水、辐照环境下的腐蚀机理; Corrosion and radiation damage of reactor materials.</p> <p>(5) 核燃料、包壳材料、堆内构件、反应堆压力容器、主冷却剂管道、蒸汽发生器传热管等核电厂主要设备及部件的选材原则、材料强化机制、在核电厂工况下的老化及失效机理、防范措施等; Materials for nuclear fuel, cladding, internals, pressure vessels, primary coolant piping, steam generators, their performances, degradation and mitigation of failure.</p> <p>(6) 先进反应堆材料的发展。Development of materials for future advance reactors.</p>
*参考书目 References	<p>(1) 自编讲义, 张乐福, 上海交通大学教材科, 电子版下载地址: ftp://lfzhang:public@public.sjtu.edu.cn/%B1%BE%BF%C6%B</p>

	<p>A%CB%B2%C4%C1%CF/.</p> <p>(2) 杨文斗,《反应堆材料学》,原子能出版社,北京</p> <p>(3) [美] Olander,D.R., Fundamental Aspects of Nuclear Reactor Fuel Elements, Technical Information Center of Public Affairs, Energy Research and Development Administration, 1976</p> <p>(4) 材料课讲义, 下载地址: <a href="ftp://lfzhang:public@public.sjtu.edu.cn/for%20masters/">ftp://lfzhang:public@public.sjtu.edu.cn/for%20masters/</a></p>
备注 Notes	